INVESTIGATION OF ⁴⁴Ti PRODUCTION FOR ⁴⁴Ti/⁴⁴Sc RADIONUCLIDE GENERATOR

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44Ti/44Sc RADYONÜKLİD JENERATÖRÜ İÇİN 44Ti ÜRETİMİNİN İNCELENMESİ

Abstract:

As a positron emitter, Scandium-44 (⁴⁴Sc) having a short half-life of 3.97 (4) h is a promising PET radionuclide which can be produced ⁴⁴Ti long-lived parent (60.0 (11) y). In this work, a ⁴⁴Ti/⁴⁴Sc generator was designed to produce ⁴⁴Sc. The separation and purification studies were optimized by using ⁴⁶Sc (83.787 (16) d) as tracer. Hydroxamate and Dowex resins were used for separation process Sc radioisotopes from titanium. Briefly a ⁴⁴Ti/⁴⁴Sc generator system may be a source of ⁴⁴Sc production in hospitals in future. If it is made suitable for hospital use, a hospital will have the opportunity to work with this generator for many years with a single production.

Özet:

Bir pozitron yayıcı olarak, 3,97 (4) h'lik kısa bir yarı ömre sahip olan scandium-44 (⁴⁴Sc), uzun ömürlü ⁴⁴Ti ana çekirdeğinden (60.0 (11) y) üretilebilen umut verici bir PET radyonükliddir. Bu çalışmada ⁴⁴Sc üretmek için ⁴⁴Ti/⁴⁴Sc jeneratör sistemi tasarlanmıştır. Ayırma ve saflaştırma çalışmaları, izleyici olarak ⁴⁶Sc (83.787 (16) d) kullanılarak optimize edildi. Titanyumdan Sc radioizotopları ayırma işlemi için hidroksamat ve dowex reçineleri kullanılmıştır. Kısaca ⁴⁴Ti/⁴⁴Sc jeneratör sistemi gelecekte hastanelerde ⁴⁴Sc üretim kaynağı olabilir. Hastane kullanımına uygun hale getirilirse bir hastane tek üretim ile bu jeneratör ile uzun yıllar çalışma imkanına sahip olacaktır.

Keywords: ⁴⁴Sc, ⁴⁴Ti, ⁴⁴Ti/⁴⁴Sc generator, cyclotron, ⁴⁵Sc(p,2n)⁴⁴Ti, hydroxamate resin. **Anahtar Kelimeler:** ⁴⁴Sc, ⁴⁴Ti, ⁴⁴Ti/⁴⁴Sc jeneratörü, siklotron, ⁴⁵Sc(p,2n)⁴⁴Ti, hidroksamat reçinesi.

1. Introduction

PET radionuclides are produced in cyclotron and as generator products. Recently, the production of PET radionuclides has gained an interest to use in nuclear medicine both in terms of being economical and the ease of the applied method. Currently, the search for other radioisotopes that can replace ¹⁸F, the most common PET radionuclide produced in particle accelerators, is still ongoing (Ferreira et al., 2012; Jokerst & Gambhir, 2011). It has been proven that especially the new metallic PET radionuclides have better organ involvement and therefore good solubility (Sadeghi, Enferadi & Nadi, 2011; Walczak et al., 2015). Although ⁴⁴Sc does not have a single production method, there are also different production conditions. Although this radionuclide is a generator product, it can also be produced in accelerators with different targets (Daraban et al., 2009; Hoehr et al., 2014; Severin et al., 2012). However, the general

method is often the method obtained by irradiating the natural scandium compounds of ⁴⁴Ti. Due to the economic and physical convenience of this method, studies on this subject have increased in recent years (Kerdjoudj et al., 2016a; Lee, Kong & Hur, 2016; V. Radchenko et al., 2016; Valery Radchenko et al., 2017; Wittwer et al., 2011). As a result of loading the main radioisotope into the system and elution with certain chemicals, this process takes seconds. Another feature of the generators is that they can be used for much longer periods than the half-life of the product radionuclide, depending on the half-life of the main radionuclide. Since the half-life of ⁴⁴Ti, the main radionuclide, is approximately 60 years in this project, the use of a single generator will be possible for many years (Ayranov & Schumann, 2010; Filosofov, Loktionova, & Rösch, 2010; Lange et al., 1999; Pruszyński et al., 2010; Roesch, 2012). This is of great importance in terms of ease of use and economy.

With its long half-life, ⁴⁴Ti is promising as a ⁴⁴Ti/⁴⁴Sc generator. However, the production of ⁴⁴Ti can be expensive because the irradiation time to reach higher activities is long. There are two primary production methods of ⁴⁴Ti: the first is ${}^{45}Sc(p,2n){}^{44}Ti$ reaction in particle accelerators (Daraban et al., 2009; Yug et al., 2005), and the second is using ⁴⁰Ca with alpha particles (Alliot et al., 2015; Severin et al., 2012; Szkliniarz et al., 2016). For the first method, the optimum parameters for irradiation are 22 MeV proton acceleration for 10 days in a cyclotron. Yet, with these parameters, only about 1 mCi is achieved because of the low crosssection values. After production, a long cooling time is also needed due to the generation of other products. Nevertheless, such other products are negligible because of their short half-lives and are easily separated using resin systems (Valery Radchenko et al., 2017). After the separation of ⁴⁴Ti using this method, the ⁴⁴Ti/⁴⁴Sc generator can be produced using known ways, e.g., as for a ⁹⁹Mo/^{99m}Tc generator (Filosofov et al., 2010; Kerdjoudj et al., 2016b; Pruszyński et al., 2010; Valery Radchenko et al., 2017; Roesch, 2012). For the second method, when ²²⁶Ra is used as an alpha particle on Ca targets, ²²⁶Ra can decay with the same energy levels to ⁴⁴Ti gamma energy levels. Thus, the separation of ⁴⁴Ti and other products from ²²⁶Ra (such as ²¹⁴Pb) is essential. Of note, ⁴⁴Ti and ²¹⁴Pb are not easily separable with gamma spectroscopy.

2. Material and Method

2.1. Reagents and equipment

As for the chemicals used in this study, thin layer chromatography paper (ITLC-silica), TiCl₄, CaCl₂, methanol, hydrochloric acid, 2,3,5,6-tetrafluorophenol (TFP), oxalic acid, sodium hydroxide (NaOH, pH adjuster), acetonitrile were purchased from Merck Chemical Co. (Darmstadt, Germany); CaCl₂, Dowex 1X8, 8-hydroxyquinoline, EDTA (ethylenediaminetetraacetic acid), 2,3,5,6-tetrafluorophenol (TFP), and N-(3-dimethylamino propyl)-N'-ethyl carbodiimide hydrochloride (EDC) were supplied from Sigma Aldrich (Darmstadt, Germany); and Accell resin in the Sep-Pak cartridge was purchased from Waters Co. (Milford, USA).

The following equipment from Ege University Institute of Nuclear Sciences was used: high-performance liquid chromatography (HPLC) SPD-10AV UV/vis and NaI (Tl) scintillation gamma detector and diode array detector (DAD) (Shimadzu SPD-M20A) systems with LC-10Atvp pump (Shimadzu Corporation, Kyoto, Japan), SIL-20A HT automatic sampler (Shimadzu Corporation, Kyoto, Japan), Inertsil ODS-3 C-18 4.6 × 250 mm HPLC 5 μ m column (G.L. Sciences Inc., Tokyo, Japan), column oven (Shimadzu CTO-10ASvp), AR-2000 radio TLC imaging scanner (Eckert & Ziegler, Berlin, Germany), 1024-channel multichannel gamma spectrophotometer with 1 × 1 inch LaBr3(Ce) Detector (ORTEC), and ²²⁶RaCl₂ standard.

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2.2. Separation of ⁴⁶Sc⁺³ and Ti⁺⁴

The separation of radioactive $[{}^{46}Sc]~Sc^{+3}$ from Ti^{4+} was achieved using either hydroxamate or Dowex resin

2.3. ⁴⁶Sc⁺³ and Ti⁴⁺ Separation Using Hydroxamate Resin

Hydroxamate resin was prepared by functionalizing the carboxy groups of silica-based weak cation exchanger resin. Initially, Accell resin (0.42 g) was soaked in 8.0 mL of water in a 15 mL falcon tube for approximately one day. 30 µL of 3 M HCl, a fresh solution of TFP (0.8 g) in 250 µL of acetonitrile, and EDC (0.8 mmol) were added. The reaction mixture was mixed for one hour at room temperature using a magnetic stirrer, and then the reaction was continued by mixing for another 3 hours at room temperature. Afterward, the resin was separated by filtration and washed three times with 10 mL of water and then three times with 10 mL of acetonitrile to separate unreacted impurities. The resin containing TFP ester groups was converted into hydroxamate resin by reacting with hydroxylamine in the next step. After 0.01 mol, 694.9 mg of hydroxylamine hydrochloride was dissolved in a mixture of 1 mL of 1.0 M NaOH and 2 mL of methanol. The pH was adjusted to 5.3–5.4 with 25–50 µL of 1.0 M NaOH. Then, the solution was added to the activated resin in a 15 mL falcon tube, and the reaction (pH 5.0–5.2) was continued at room temperature for 18 hours using a magnetic stirrer. The product was purified by filtration and washed five times with 10 mL of water and 10 mL of acetonitrile, then dried in vacuum. 333 mg of dried resin was placed in an empty syringe. Then, the column was activated by washing with 8 mL of acetonitrile, 15 mL of water, and 2 mL of 2.0 M HCl, respectively. The Ti⁴⁺ and ${}^{46}Sc^{+3}$ mixture was passed through the prepared resin (200 µL) and its radioactivity was counted.

2.4. ⁴⁶Sc⁺³ and Ti⁴⁺ Separation Using Dowex 1X8 Resin

Here, 10^{-3} M ScCl₃ and 10^{-6} M TiCl₄ were dissolved in 5 mL of methanol and mixed in equal volumes. Then, 46 Sc ${}^{+3}$ was added, and the resulting solution was saturated with 5 mg of 8-hydroxyquinoline. The resin was activated before the prepared solution was passed through Dowex 1X8 resin. For this, a mixture containing 10% NaCl and 0.2% NaOH was heated at 80 °C for 2 hours, passed through the resin, and the resin was activated by passing through 0.5% HCl immediately afterward.

The mixture of ${}^{46}Sc^{3+}$ and TiCl₄ passed through Dowex 1X8 resin was examined with an ORTEC 1024-channel multichannel gamma spectrophotometer with a 1 × 1-inch LaBr₃(Ce) detector. At the first, background activity was measured. Then, background and elution spectra were determined. 5 mg of 8-hydroxyquinoline solution and ${}^{46}Sc^{3+}$ and TiCl₄ solutions were mixed and passed through Dowex 1X8 resin. The radioactivity of the eluents was counted on a multichannel analyzer. Then, 200 µL of the elution profiles were obtained by passing through 12 M HCl and their radioactivity counts were determined using a multichannel analyzer. In the last step, the count and gamma spectrum of the 200 µL sample taken from the elution obtained by passing 2 M HCl were determined. In addition, HPLC analyzes of ${}^{46}Sc^{3+}$ and Ti⁴⁺ separated by passing through Dowex 1X8 resin were performed.

3. Results and Discussion

Szkliniarz reported that ⁴³Sc, ^{44g}Sc, and ^{44m}Sc were produced by irradiating alpha particles accelerated in the cyclotron, using Ca and natural K targets, and highly pure ⁴³Sc was obtained (Szkliniarz et al., 2016). Cyclotrons are known to be economical and advantageous. Several studies of nuclear reactions with deuterium and protons in cyclotrons have been

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conducted (Tárkányi et al., 2019). While ⁴⁴Sc can be produced directly with isotopes of ⁴³Ca and ⁴⁴Ca, we hypothesized that it is possible to produce a ⁴⁴Ti/⁴⁴Sc generator by creating ⁴⁴Ti by reacting ⁴⁵Sc with protons or deuterons. The disadvantages of ⁴⁴Ti that we produced directly under our working conditions compared to cyclotron production are its lower particle energy, lower particle density, and the impurities of ²²⁶Ra from other radionuclides in the decay chain. Our results are particularly valuable for individuals or groups that do not have the opportunity to work with a cyclotron within an academic environment. Table 1 presents possible reactions for ⁴⁴Ti production expect ⁴⁵Sc.

Reaction Products	Q-value (keV)	Threshold Energy (keV)
⁴⁰ Ca(α,γ) ⁴⁴ Ti	5.127	0
⁴⁸ Ti(p,2n+t) ⁴⁴ Ti	-36.747	35.477
⁴⁸ Ti(d,3n+t) ⁴⁴ Ti	-36.972	38.525
$^{51}V(p,4n+\alpha)^{44}Ti$	-42.076	42.908
⁴⁸ Ti(n,5n) ⁴⁴ Ti	-43.229	44.139
⁵¹ V(d,5n+α) ⁴⁴ Ti	-44.301	46.052
⁵¹ V(n,5n+t) ⁴⁴ Ti	-61.890	63.116

In these reactions, the simplest isotope, which can be worked with, is 40Ca. Some of the by-products that can be formed according to the ${}^{40}Ca(\alpha,\gamma){}^{44}Ti$ reaction are ${}^{44}Ti(\alpha,p){}^{47}V$, ${}^{44}Ti(\alpha,\gamma){}^{48}Cr$, ${}^{40}Ca(\alpha,p){}^{43}Sc$, ${}^{43}Sc(p,\gamma){}^{44}Ti$, ${}^{44}Ti(p,\gamma){}^{45}V$, and ${}^{45}V(p,\gamma){}^{46}Cr$. Based on these results, there was no chemical or radiochemical damage of any by-product. Gamma energies of the ${}^{226}Ra$ isotope (used as an alpha source) and of its decay products are presented in Table 2.

Radioisotope	Gamma Energy (keV)	Half-life
²²⁶ Ra	81.07	1602 years
	83.78	
	186.1	
	600	
²¹⁰ Pb	47	21 years
²¹¹ Pb	405	36.1 minutes
	832	
²¹² Pb	239	10.64 hours
²¹⁴ Pb	74.81	26.8 minutes
	77.11	
	295	
	352	
	766	
	785	
²²² Rn	186	3.8 days
²¹⁴ Po	609	162.3 µseconds
	665	
	768	
	934	
	1120	
	1238	
	1764	
	2118	
²¹⁴ Bi	241	19.9 minutes
	295	
	351	
	785	
⁴⁰ K	1460	10 ⁹ years

Table 2. Gamma energies after ²²⁶Ra decay chain (Chisté, Bé, & Dulieu, 2007).

Gamma energies for the 44 Ti isotope and reaction by-products after 226 Ra decay are presented in Table 2.

Radioisotope	Gamma Energy (keV)	Half-life
⁴⁴ Ti	67.8	60 years
	78.3	
⁴⁶ Sc	889	83.9 days
	1120	
⁵² V	1434	3.76 minutes
⁴⁷ Ca	1308	4.53 days
²² Na	511	2.60 years
	1274	
²⁴ Na	1369	15 hours
³⁸ C1	1600	37.3 minutes
47V	511	32.6 minutes
⁴⁸ Cr	511	21.6 hours
45V	511	547 mseconds
⁴⁶ Cr	511	0.26 seconds
⁴⁴ Sc	511	3.97 hours

Table 3. Gamma energies and half-lives of the ⁴⁴Ti isotope and other primary products after ²²⁶Ra alpha particle's reaction with ⁴⁰Ca.

3.1 Results of Separation with Hydroxamate Resin using ⁴⁶Sc radiotracer

In this study, we achieved the separation of ${}^{46}Sc$ from the target system using hydroxamate and Dowex resins. The results show that hydroxamate resin is suitable for separating ${}^{46}Sc$ from the target. With 6 M HCl and hydroxamate resin, 97.4% of the total radioactivity of ${}^{46}Sc$ was separated. These results are in line with those reported previously by Gagnon et al., who used hydroxamate resin previously for the separation of ${}^{45}Sc$ and who reported that the extraction of ${}^{45}Ti$ was possible with > 50% recovery in 1 mL of 1 M oxalic acid (Gagnon et al., 2012). They also reported that this resin could also be used successfully in ${}^{89}Zr$ separation. Indeed, in our previous study, the hydroxamate resin was used for the production of ${}^{89}Zr$, whereby 75% of ${}^{89}Zr$ was eluted from the hydroxamate resin with oxalic acid (Bulduk et al., 2019).

3.2 HPLC Analysis of the Dowex 1X8 Resin, ScCl₃, and TiCl₄ Separations

⁴⁶Sc, a relatively long-lived radionuclide that decays by ß and γ radiation with a halflife of 83.8 days of Sc, was used as a carrier to determine Sc separation and purification of ⁴⁶Sc.

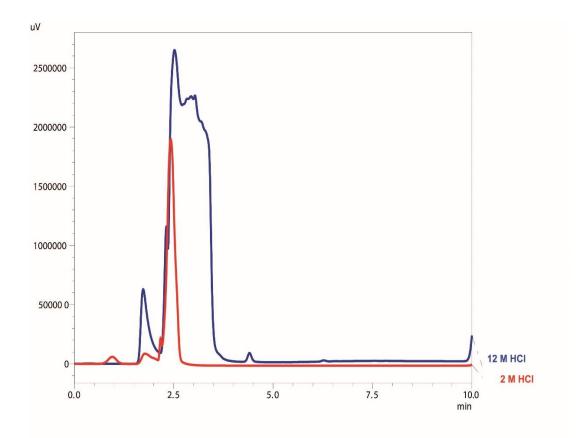


Figure 1. ⁴⁶Sc elution profile passing through Dowex 1X8 Resin (activity as a percentage).

Figure 1 shows that ⁴⁶Sc radioactivity can be eluted with 12 M HCl at a rate of 93.86% with Dowex 1X8 resin. Our results agree with Bartos et al. who separated ⁴⁷Sc with 0.5 M ammonium acetate by adsorbing it on Dowex 50 cation resin. The ⁴⁷Sc separation efficiency in the proposed procedure is about 90%, with a separation time of fewer than 2 hours. The resulting carrier-free ⁴⁷Sc was used to label the DOTATATE conjugate (Bartoś et al., 2012).

4. Conclusion

An in-house ${}^{44}\text{Ti}/{}^{44}\text{Sc}$ generator can be highly valuable for hospitals, making profound contributions to the health system. In addition, such a generator affords the convenience of working with a single generator for many years thanks to the long half-life of ${}^{44}\text{Ti}$, dramatically reducing the costs of producing ${}^{44}\text{Sc}$.

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